

Worldwide Development of Degradation Resistant Nuclear Materials Triggered by the Fukushima Nuclear Accident

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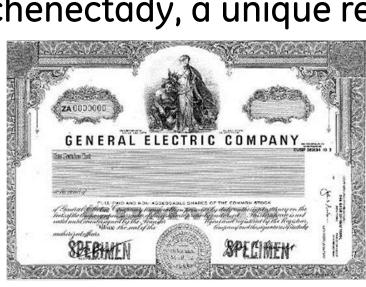
General Electric

- 1) Started in Schenectady, NY in 1892
- 2) Founded by Thomas A. Edison
- 3) Operates in 180 countries in the world
 - Known by Aviation, Power, Oil & Gas, Healthcare, etc.

4) GE Global Research Center in Schenectady, a unique research center

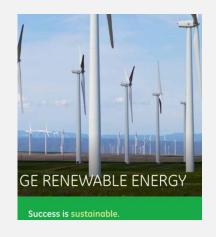
in the world. Started in 1900.

5) 1100 engineers (685 PhDs)

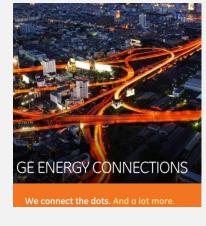




GE Today

















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Driving technology advantages across our businesses





Accelerating innovation through collaboration





The accident and the opportunities



11-March-2011, Tsunami at Fukushima Nuclear Site

- The second wave 15 m (50 ft) high hit the turbine buildings, destroying the diesel generators in the first floor and in the basement.
- Plant black out. No coolant for the fuel rods in the reactor or the spent fuel in the pools.







The second wave, 11 March 2011

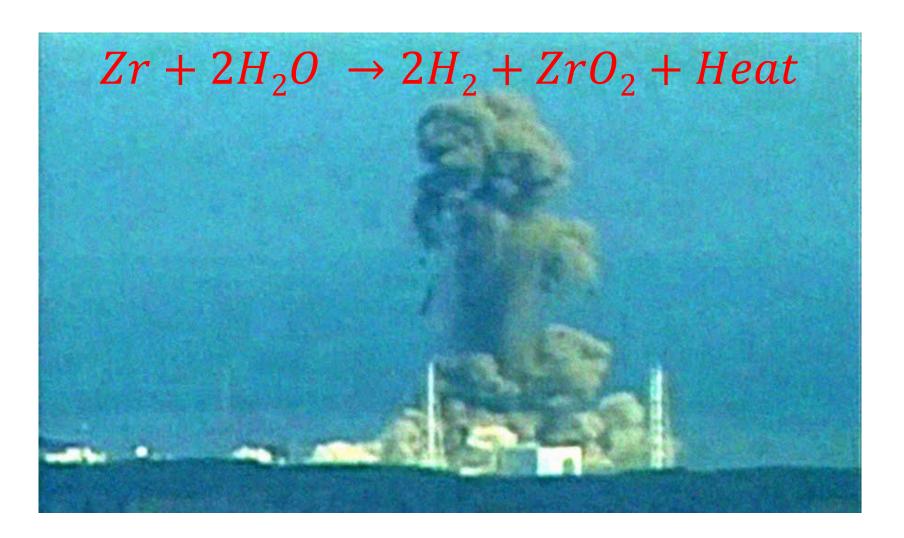




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Building explosions due to the ignition of accumulated hydrogen gas





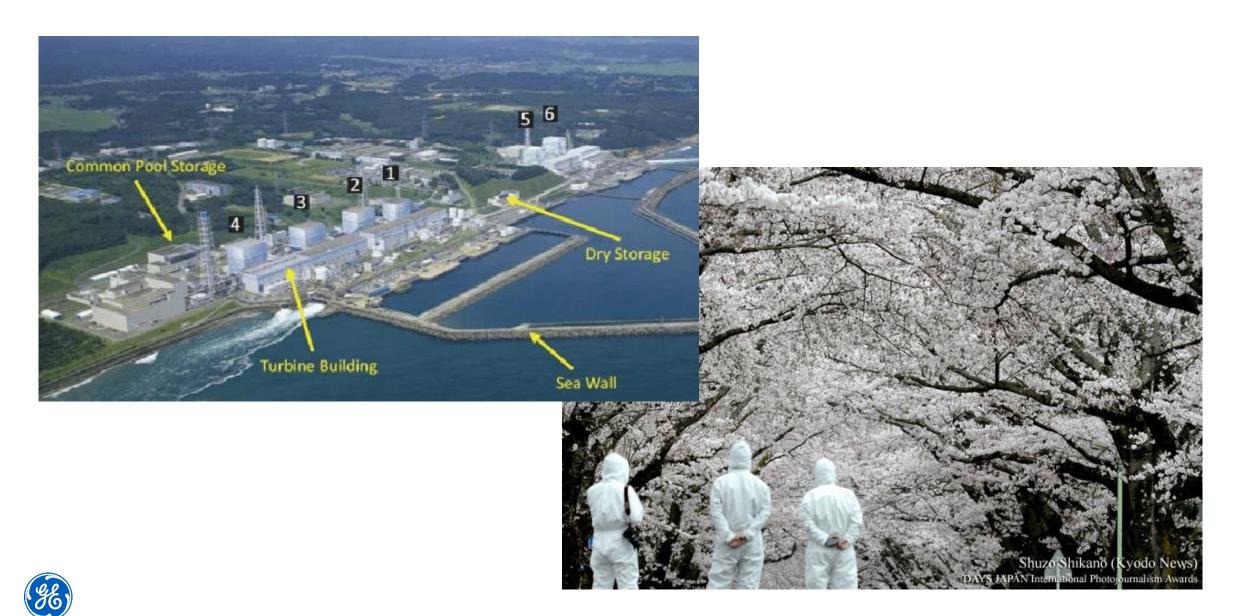
Fukushima Daiichi Reactors after the Explosions





View from the Sea

Fukushima Daiichi – Pre Tsunami.



Fukushima Daiichi Recent View





In Chinese, <u>Crisis</u> = wēi jī





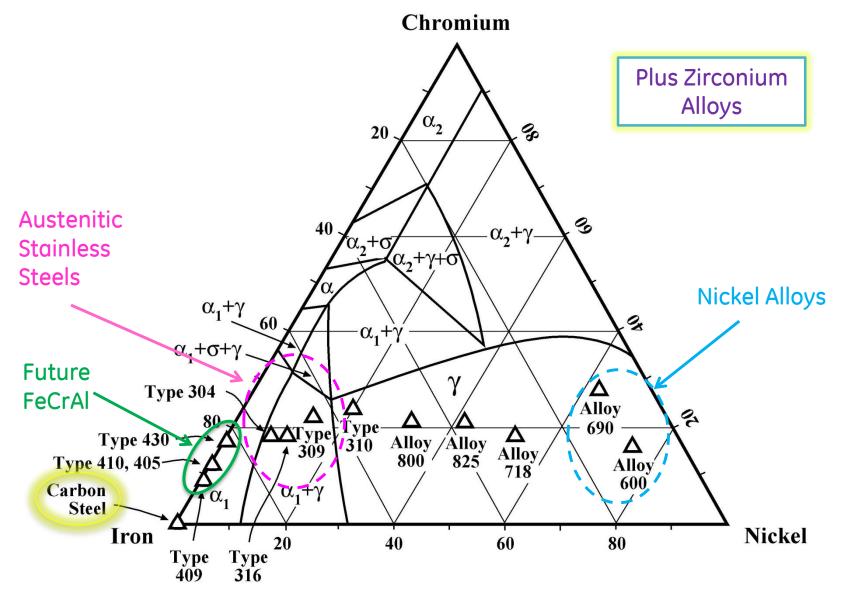




Materials in Nuclear Power



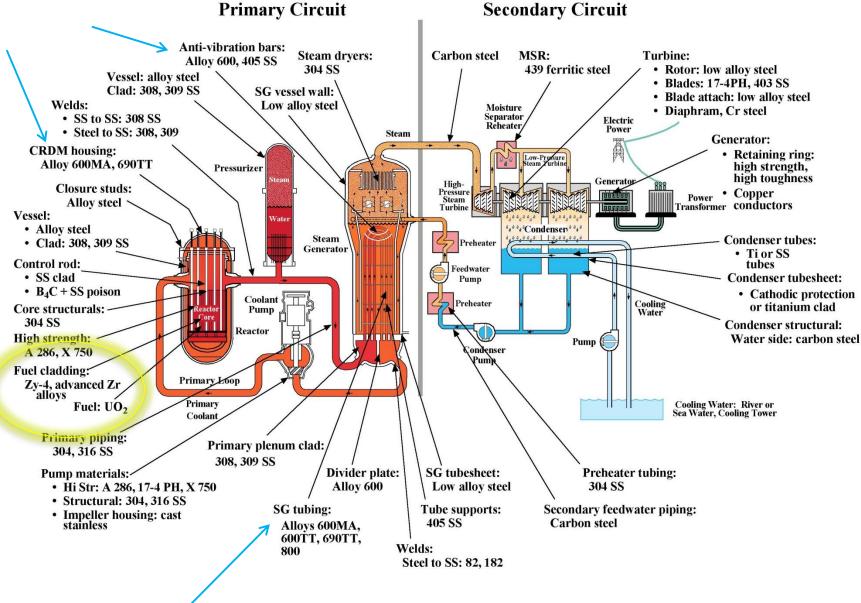
Structural Materials in the Nuclear Power Industry





Source: R. W. Staehle

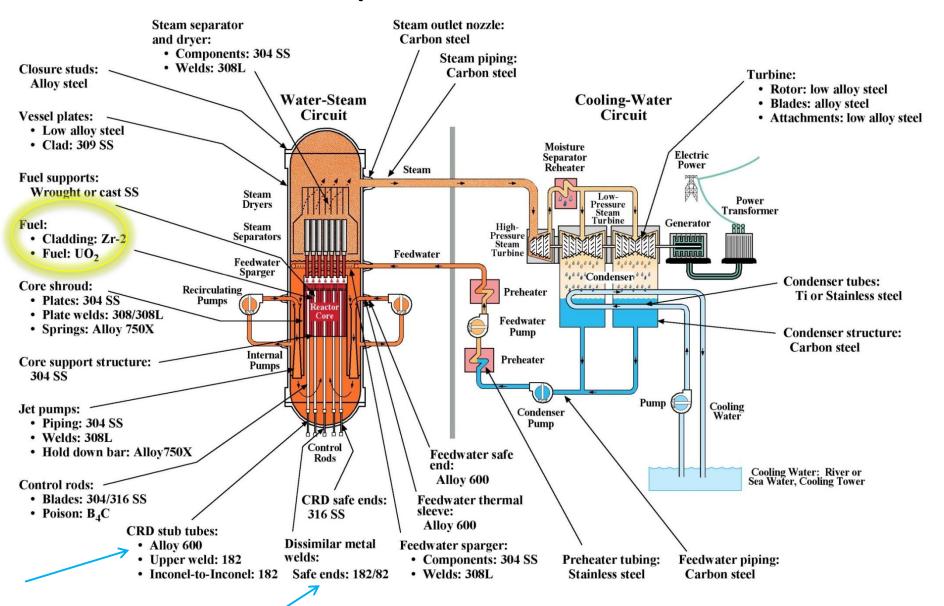
Materials and Components in PWRs





Source: R W Staehle

Materials and Components in BWRs





Source: R. W. Staehle

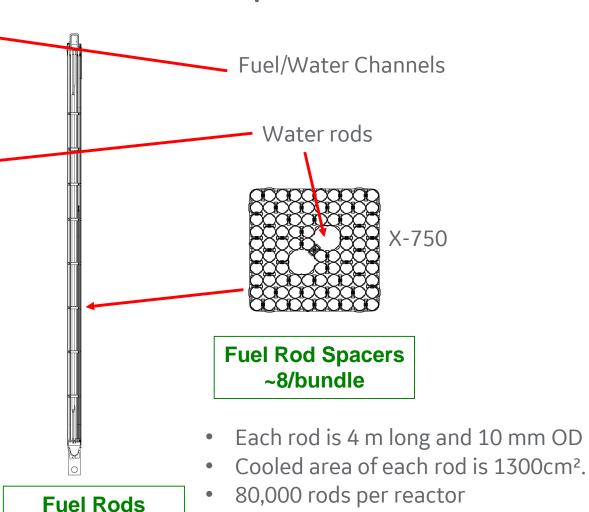


Department of Energy Accident Tolerant Fuels



Water moves up **Fuel Bundle** BWR 6 ~800/core ~1300MWe

Fuel Components in BWR



~80-100/bundle



Accident Tolerant Fuels (ATF) Advanced Technology Fuels (ATF)

Improved Reaction Kinetics with Steam

- Heat of oxidation
- Oxidation rate

Improved Fuel Properties

- Lower operation temperatures
 - Clad internal oxidation
- Fuel densification/relocation
 - Fuel melting



Retention of Fission Products

- Gaseous fission products
- Solid/liquid fission products

Improved Cladding Properties

- Clad fracture
- Dimension stability
- Thermal shock resistance
 - Clad melting

Slower Hydrogen Generation Rate

- Hydrogen bubble
- Hydrogen explosion
- Hydrogen embrittlement of clad





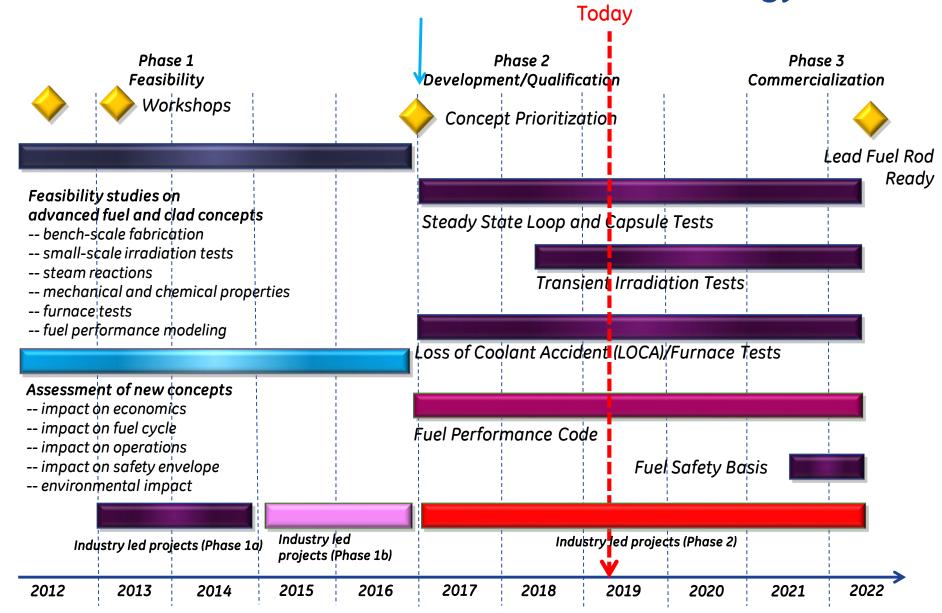
Department of Energy – Nuclear Energy

Since October 2012, DOE-NE has supported the research of newer materials for nuclear power plants providing cost shared funding.

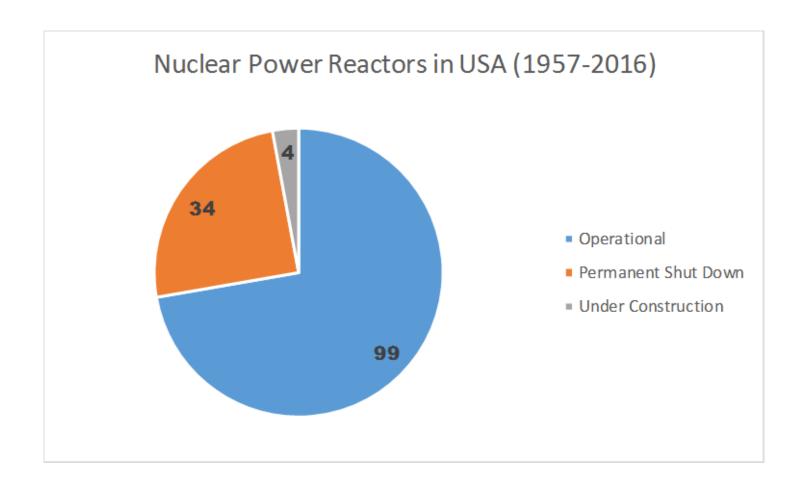
Companies supported include

- 1) Framatome Coatings of Zircaloy
- 2) General Electric FeCrAl cladding, SiC
- 3) Westinghouse Coatings of Zircaloy, SiC

Ten-Year ATF Plan from DOE Nuclear Energy

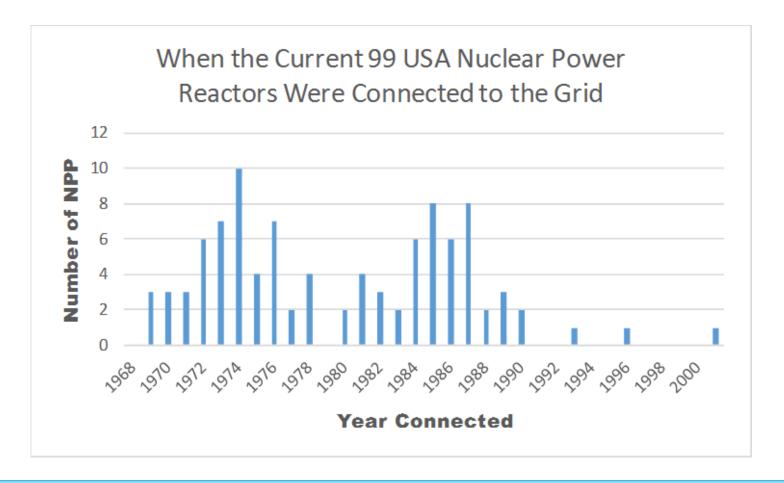


Nuclear Power Reactors in the USA 1957-2016





89% of the Reactors are at least 30 years old



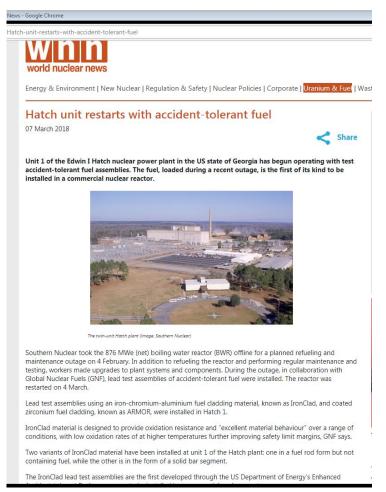
To implement Accident Tolerant Fuels in the current reactor fleet, it needs to be a concept with near term solution.

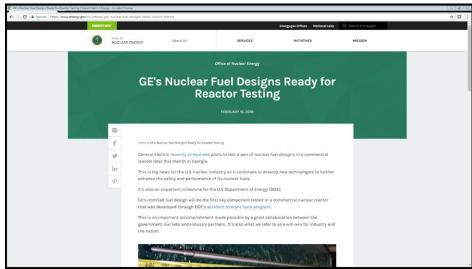


IronClad (FeCrAl) - GE/ORNL Effort, February 2018

Southern Nuclear Initial Installation – Plant Hatch, Unit 1 Cycle 29



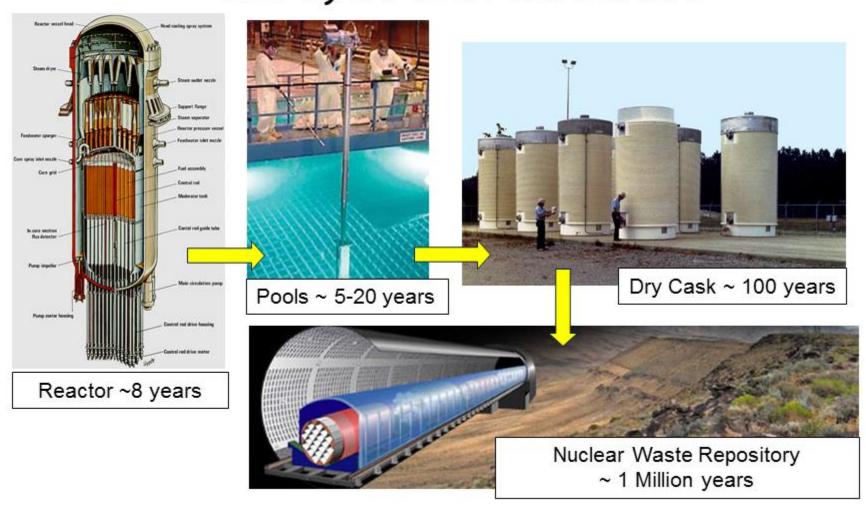






The Fuel Cycle

Life Cycle of a Fuel Bundle



The GE Accident Tolerant Fuel Team

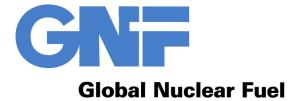


GE Global Research



HITACHI

















University of Idaho



Why FeCrAl Alloys for Cladding?



Development of High Performance FeCrAl Fuel Cladding for Light Water Power Reactors

FeCrAl cladding concept:

Fe + 10/22 Cr + 4/6 Al (+ 3 Mo, Hf, Y, Zr, etc.)

APMT = Powder Metallurgy

C26M = Traditional Melting

NFD ODS = Powder Metallurgy from Japan

Near term solution for <u>enhanced safety of current light water reactors</u>.

- 1. Superior oxidation resistance in the event of a severe accident.
- 2. Excellent environmental characteristics under normal operation both for BWR and PWR coolants
- 3. Eliminates common current fuel failure mechanisms
- 4. No change in fuel type (Utilizes current UO₂ fuel)

Needed Cladding Performance for

Normal Operation Conditions

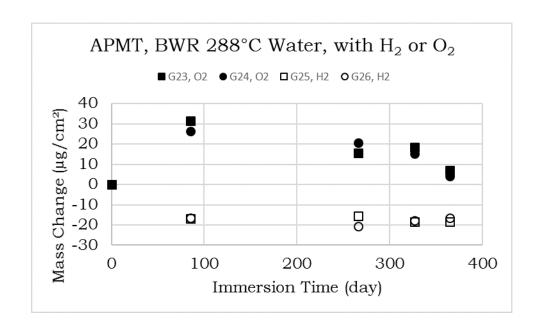
- Boiling Water Reactors, 288°C, water + hydrogen or oxidants
- Pressurized Water Reactor, 330°C, water + hydrogen

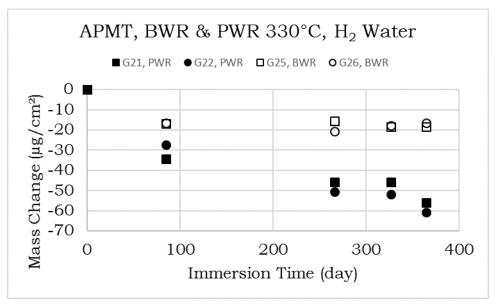
Severe Accident Conditions

- Loss of Coolant Accident
- Steam or steam + air at T > 1200°C



Low general corrosion under normal operation conditions.





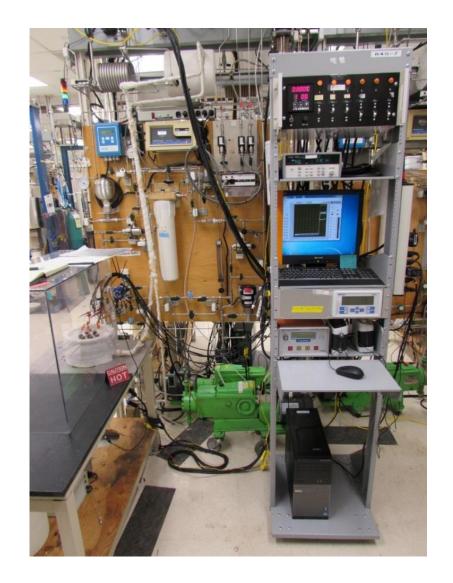


Normal Operation Electrochemical Behavior

Element	Zircaloy-2 hcp	304SS austenitic	X-750 austenitic	C26M ferritic	APMT ferritic
Zr	~98				
Fe	0.07-0.2	~70	7	~80	~70
Ni	0.03-0.08	8	~72		
Cr	0.05-0.15	19	15	12	22
Al			0.7	6	5
Мо				2	3
Si		0.75 max	0.5 max	0.2	0.7 max
Mn		2 max	1 max		0.4 max
С		0.08 max	0.08 max	0.01 max	0.08 max
Other	1.5Sn		2.5Ti, 1Nb	0.05Y	Y, Hf

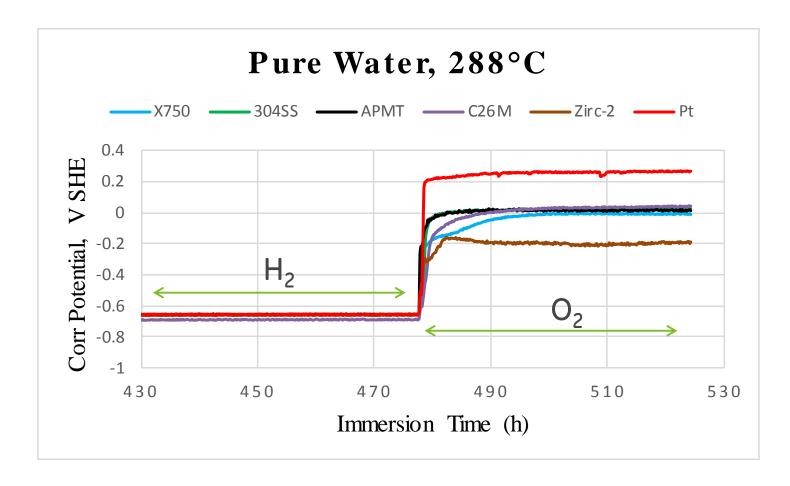
Corrosion potential vs. immersion time in pure water at 288°C, containing either 0.3 ppm hydrogen or 1 ppm oxygen.

Electrochemical Testing in 288°C Water



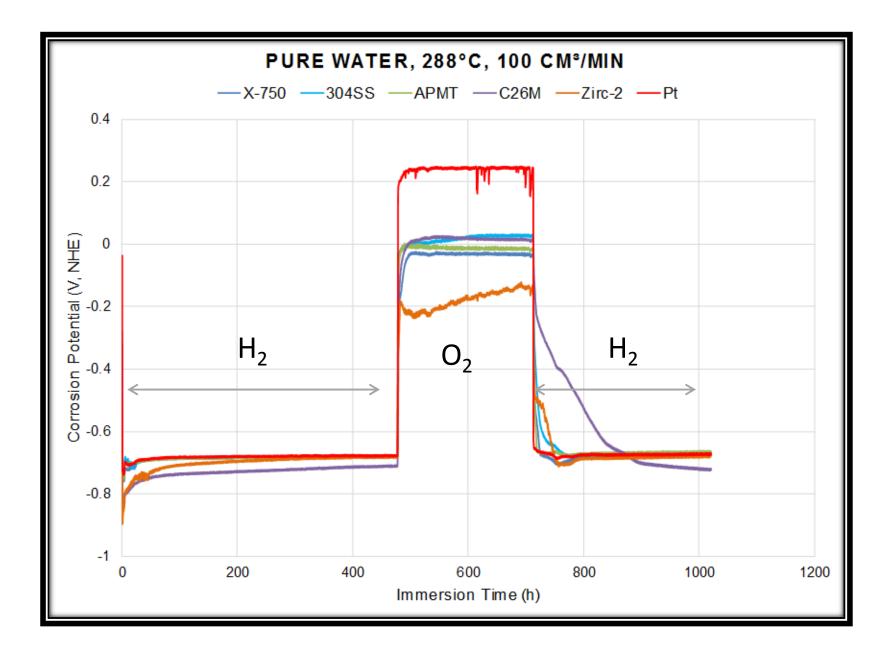


Normal Operation Electrochemical Behavior



Corrosion potential vs. immersion time in pure water at 288C. In the first period the water contained 0.3 ppm hydrogen and in the last, 1 ppm oxygen.

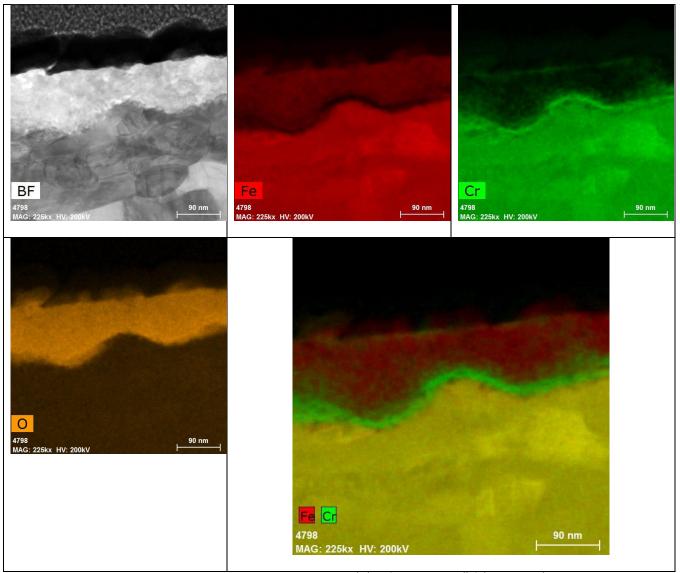
Electrochemical Behavior Normal Operation



Corrosion potential vs. immersion time in pure water at 288C. In the first period the water contained 0.3 ppm hydrogen and in the last, 1 ppm oxygen.

Normal Operation with Protective Oxide

Surface & cross-section of the oxide formed on APMT coupon exposed for 1 year to



BWR, pure water + $1 \text{ ppm O}_2 \text{ at } 288^{\circ}\text{C}$

Dual Layer Oxide

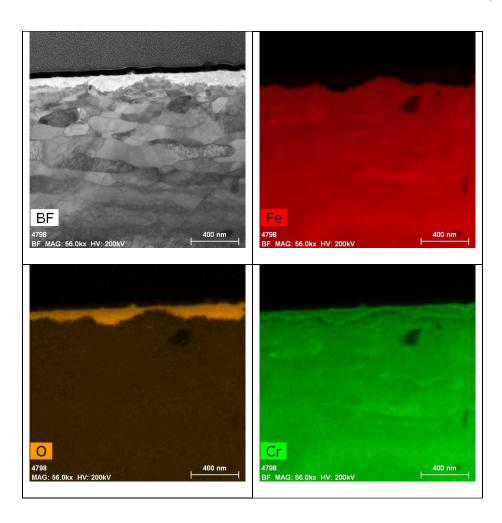
A ~10 nm thick inner layer of Cr_2O_3

No Mo or Al in the oxides.



Normal Operation PWR Condition, Protective Oxide

Surface & cross-section of the oxide formed on APMT coupon exposed for 1 year to



PWR, pure water + 3.75 ppm H₂ at 330°C

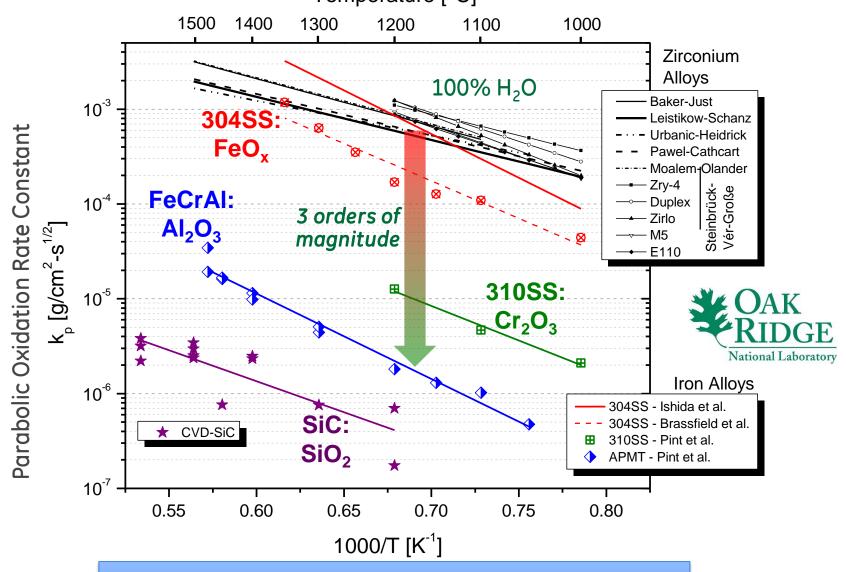
A single layer oxide rich in Cr

No Fe, Mo or Al in the oxide.



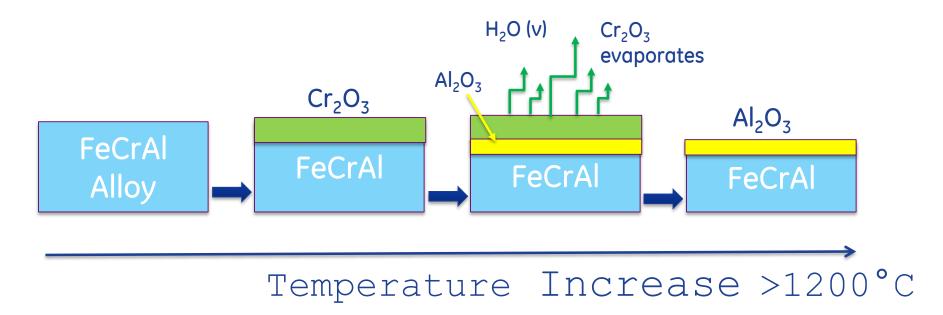
Comparison on Sperheated Steam Oxidation Rate

K.A. Terrani, S.J. Zinkle, L.L. Snead, "Advanced oxidation-resistant iron-based alloys for LWR fuel cladding," *J. Nucl. Mater.*, **448**, (2014) **Temperature** [°C]



High temperature oxidation of Fe-Cr-Al alloys.

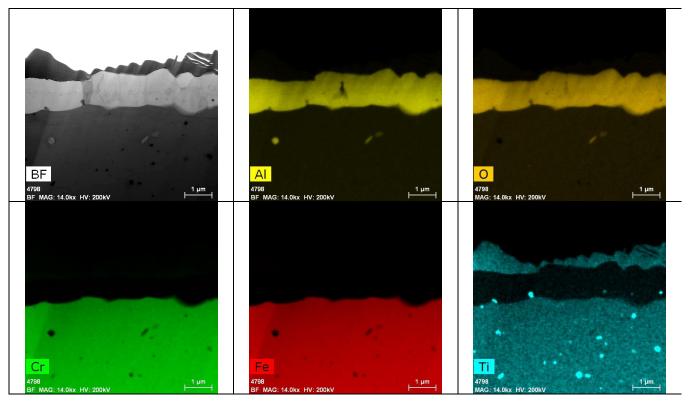
 Outstanding resistance to oxidation in steam at temperatures higher than 1000°C



- a) Up to near 1000°C, Cr₂O₃ protects the material from further oxidation
- b) At 1200°C and higher steam temperatures, FeCrAl protected by an alumina layer

Alumina layer forms on the surface of APMT Tube, 0.8 mm wall thickness and 9.5 mm OD after exposure to 100% steam at 1200°C for 4 h

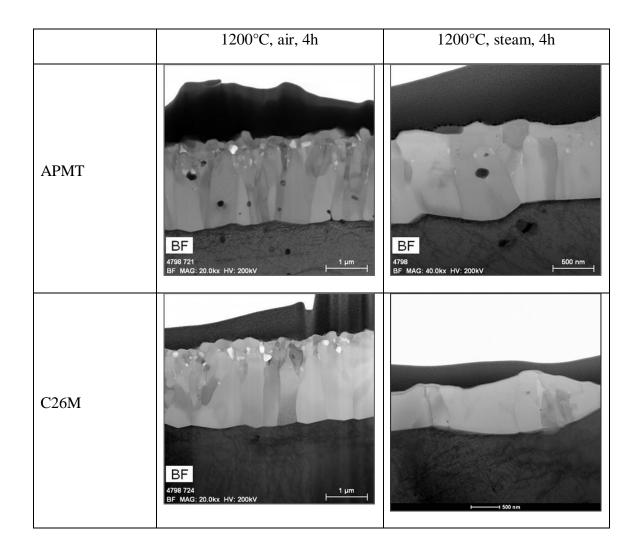
~1 µm thick Alumina Layer



~70Fe + 21Cr + 5Al + 3Mo

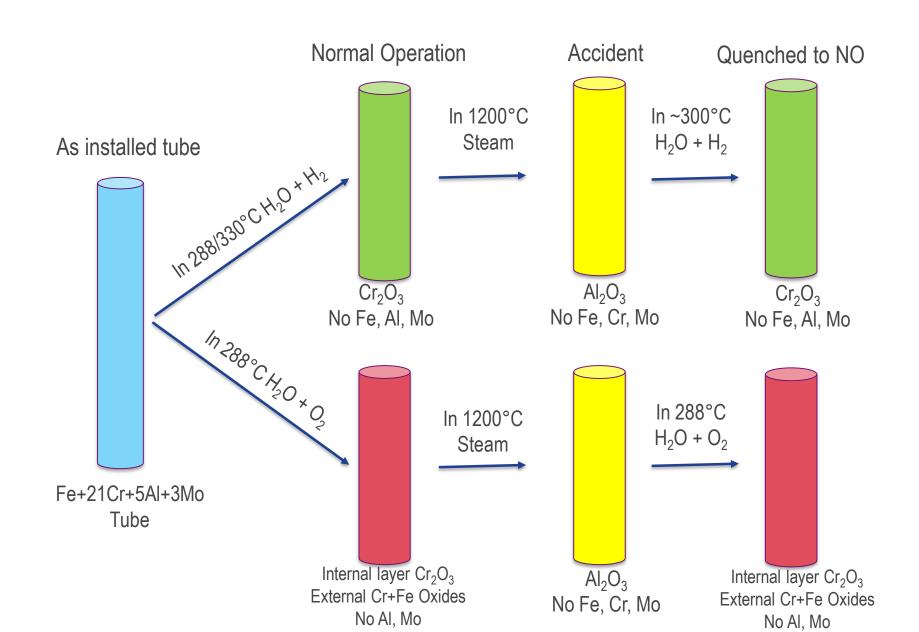


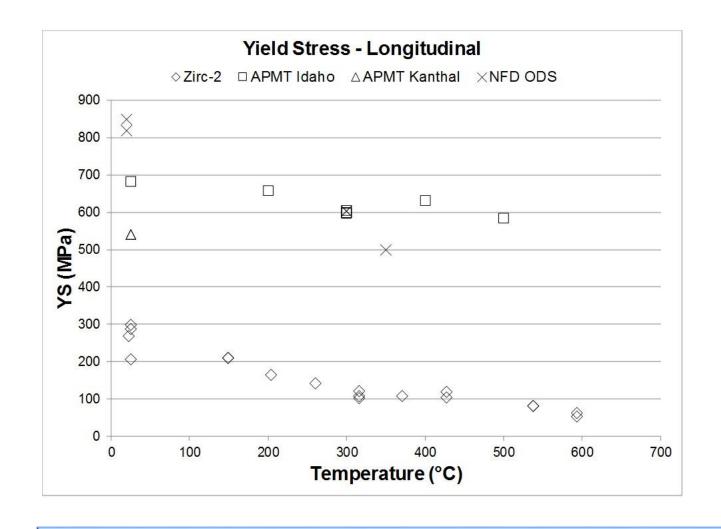
At 1200°C, the oxide was thinner in steam than in air.



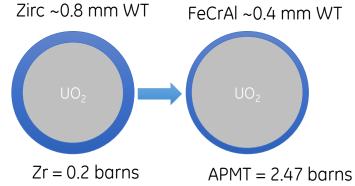


Surface Oxide Evolution on APMT FeCrAl



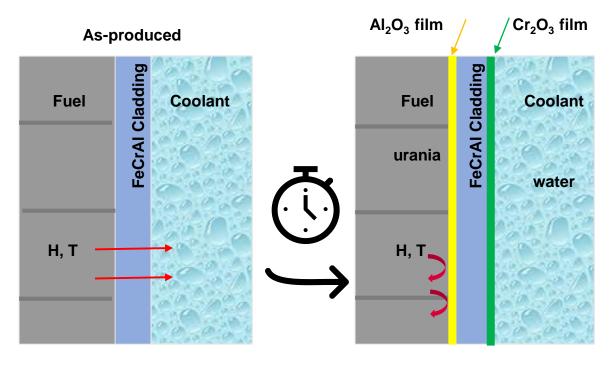


Non-Irradiated Mechanical Properties up to 600°C



Higher initial IronClad strength favorable for fuel performance allows for thinner wall of the cladding.

Radioactive Hydrogen forms in the Fuel and May Migrate through the Cladding Wall into the Coolant.



FeCrAl is ferritic, atomic hydrogen permeates through the wall. Hydrogen do not form stable hydrides with any of the elements of the cladding. Surface oxides will minimize hydrogen permeation through the cladding wall.



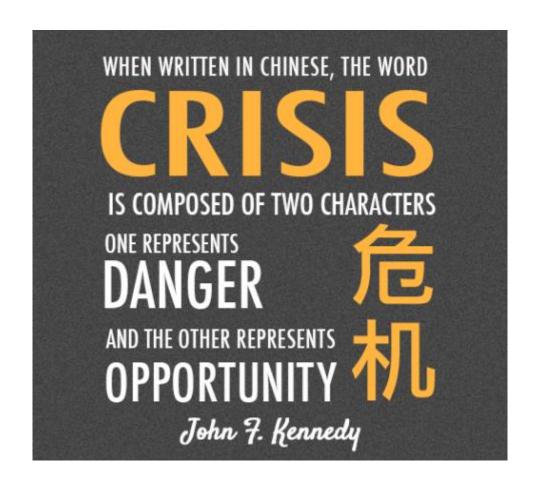
Conclusions



Summary and Conclusions

- 1) The Fukushima Daiichi accident in March 2011 was an unfortunate event; however it re-energized the materials development for nuclear power plants.
- 2) General Electric (GE) and Oak Ridge National Laboratory (ORNL) are pursuing an accident tolerant fuel (ATF) design concept named IronClad which utilized FeCrAl material for cladding and uranium dioxide (UO₂) fuel pellets.
- 3) The use of IronClad will greatly improve the safety of plant operation by putting a sturdier primary barrier between the radioactive elements in the fuel and the coolant.
- 4) The use of IronClad cladding is an advanced ATF concept, which could be implemented in the <u>current fleet</u> of the light water power reactors.
- 5) GE/GNF and Southern Nuclear completed in February 2018 the first ATF installation of non-fueled rods in a commercial reactor (Hatch-1) for the entire DOE program.

Crisis = Danger & Opportunities



[The word] "crisis" (wēijī) consists of two syllables that are written with two separate characters, wei and jī. While it is true that weiji does indeed mean "crisis" and that the wēi syllable of wēijī does convey the notion of "danger," the jī syllable of wēijī most definitely does not signify "opportunity." The jī of wēijī, in fact, means something like "incipient moment; crucial point (when something begins or changes)."

